

ICSBEP FIVE-YEAR PLAN	
OAK RIDGE NATIONAL LABORATORY	
<i>IDENTIFIER</i>	<i>DRAFT TITLE</i>
<i>FY-2005</i>	
LEU-COMP-THERM-068	Plexiglas, Concrete, and Steel-reflected U(4.46)3O8 with H/U=1.25
LEU-COMP-THERM-069	Plexiglas and Concrete-Reflected U(4.46)3O8 with H/U=2.05
LEU-MET-THERM-007	Libby Johnson U(4.89) Metal Rods in Water or Uranyl Fluoride Solution
U233-COMP-THERM-004	Bettis U233-Th Lattice Physics Experiments, Judd Hardy, et.al.
U233-SOL-THERM-016	Un-reflected Solutions of ²³³ UO2(NO3)2 in Cylinders
U233-SOL-THERM-017	Water-Reflected Solutions of ²³³ UO2(NO3)2 in Cylinders
<i>FY-2006</i>	
IEU-MET-THERM-001	Cronin U(37.5) Metal Experiments, Recently Unclassified
IEU-SOL-THERM-006	Cronin UF4-CF2 from 0.2 to 37.5% U-235 (ORNL-2968)
LEU-COMP-THERM-067	Cronin Sterotex U(4.89) Blocks, H/U from 0 to 37, ORNL-2986
LEU-MET-THERM-008	Libby Johnson U(4.89) Metal Rods, Various Interstitial Absorbers
<i>FY-2007</i>	
SUB-HEU-MET-THERM-001	Research Reactor Fuel Assemblies (MURR fuel)
SUB-HEU-SOL-THERM-002	WINCO Slab Tanks with HEU Uranyl Nitrate Solution
U233-MET-INTER-001	Critical Measurements on the ²³³ U ZPPR Plates in the LANL ZEUS Assembly
MIX-COMP-INTER-004	Cooperative Analysis of ²³⁸ U MOX Experiment with LANL
<i>FY-2008</i>	
HEU-SOL-THERM-048	HEU Uranyl Fluoride Solution (82 g U/l) in Slab Arrays (ORNL/CF-56-7-148)
LEU-MET-THERM-009	Libby Johnson U(3.85) Annular Metal Billets (7.62 cm OD)
<i>FY-2009</i>	
<i>FY-2010</i>	
	Critical assemblies pertinent to reactor design & fuel cycle materials processing associated with the Generation-IV reactor concepts for nuclear energy generation, the advanced high temperature reactor concepts for hydrogen production and the space applications of nuclear energy. In this historical period, critical experiments pertinent to these applications were performed in Oak Ridge and elsewhere.